#### **AGREEMENT**

EXTENDING THE RENEWAL ARRANGEMENT OF SEPTEMBER 22, 2000
FOR THE EXCHANGE OF TECHNICAL INFORMATION
AND COOPERATION IN NUCLEAR SAFETY MATTERS
BETWEEN
THE NUCLEAR REGULATORY COMMISSION
OF THE UNITED STATES OF AMERICA
(U.S.N.R.C.)
AND
THE NUCLEAR REGULATORY AUTHORITY
OF THE SLOVAK REPUBLIC
(U.J.D.S.R.)

The Nuclear Regulatory Commission of the United States of America and the Nuclear Regulatory Authority of the Slovak Republic, acknowledging their desire to continue the "Renewal Arrangement for the Exchange of Technical Information and Cooperation in Nuclear Safety Matters Between the Nuclear Regulatory Commission of the United States of America (U.S.N.R.C.) and the Nuclear Regulatory Authority of the Slovak Republic (U.S.J.D.S.R.)" signed at Vienna on September 22, 2000, hereby agree to extend the terms and conditions of that Arrangement for another five years, with effect from September 26, 2005.

Both parties agree that Addenda "A" and "B," attached hereto, will replace those in the 2000 Arrangement. These new Addenda update the specific areas in which the U.S.N.R.C. and the U.J.D.S.R. are performing the nuclear safety research referred to in Articles I.1.d and I.2 of the Arrangement.

The present agreement will enter into force upon signature.

DONE at Vienna, Austria on this 26<sup>th</sup> day of September 2005, in the English and Slovak languages, both texts being equally authentic.

FOR THE NUCLEAR REGULATORY COMMISSION OF THE UNITED STATES OF AMERICA:

FOR THE NUCLEAR REGULATORY AUTHORITY OF THE SLOVAK REPUBLIC:

Nils J. Diaz, Chairman

Marta Ziakova, Chairperson

## Addendum "A"

# U.S.N.R.C. - U.J.D. S.R. Nuclear Safety Research Exchange Areas In Which the U.S.N.R.C. Is Performing or Sponsoring Safety Research

- 1. Reactor Vessel and Piping Integrity
- 2. Aging of Reactor Components
- 3. Reactor Equipment Qualification
- 4. Thermal Hydraulic Code Applications and Maintenance
- 5. Plant Performance
- 6. Human Performance
- 7. Core Melt and Reactor Coolant System Failure
- 8. Reactor Containment Safety
- 9. Containment Structural Integrity
- 10. Seismic Safety
- 11. Probabilistic Risk Assessment
- 12. Severe Accident Analysis
- 13. Radiation Protection and Health Effects
- 14. Radionuclide Transport and Waste Management
- 15. Nuclear Fuel Analysis
- 16. Dry Cask Storage and Transport
- 17. Decommissioning
- 18. Advanced Reactor Designs
- 19. Fire Protection

### Addendum "B"

## U.S.N.R.C. - U.J.D. S.R. Nuclear Safety Research

## Areas in Which the U.J.D. S.R. Is Performing Safety Research

Methods of the decision-making process for the licensing of inspection systems operation and safety systems control including risk informed regulation

Inspection of the active core power distribution and evaluation of limits for assessment of observed parameters

Methods of spent fuel safety assessment after various ways of processing

Parameters of the packed waste forms and their influence on the long-term safety of the storage area

Processes of an activity inventory and mass balance evaluation during the decommissioning of nuclear installations

Feedback and experience on safety in nuclear power plant operation

Methods of special skills verification of nuclear power plant personnel

Development and integration of the RODOS system to the emergency planning and emergency management in the Slovak Republic